



Designation: C1268 – 23

Standard Test Method for Quantitative Determination of ^{241}Am in Plutonium by Gamma-Ray Spectrometry¹

This standard is issued under the fixed designation C1268; the number immediately following the designation indicates the year of original adoption or, in the case of revision, the year of last revision. A number in parentheses indicates the year of last reapproval. A superscript epsilon (ϵ) indicates an editorial change since the last revision or reapproval.

1. Scope

1.1 This test method covers the quantitative determination of ^{241}Am by gamma-ray spectrometry in plutonium nitrate solution samples that do not contain significant amounts of radioactive fission products or other high specific activity gamma-ray emitters.

1.2 This test method can be used to determine the ^{241}Am in samples of plutonium metal, oxide and other solid forms, when the solid is appropriately sampled and dissolved.

1.3 The values stated in SI units are to be regarded as standard. Additionally, the non-SI units of electron volts, kiloelectron volts, and liters are to be regarded as standard. No other units of measurement are included in this standard.

1.4 *This standard does not purport to address all of the safety concerns, if any, associated with its use. It is the responsibility of the user of this standard to establish appropriate safety, health, and environmental practices and determine the applicability of regulatory limitations prior to use.*

1.5 *This international standard was developed in accordance with internationally recognized principles on standardization established in the Decision on Principles for the Development of International Standards, Guides and Recommendations issued by the World Trade Organization Technical Barriers to Trade (TBT) Committee.*

2. Referenced Documents

2.1 ASTM Standards:²

C758 Test Methods for Chemical, Mass Spectrometric, Spectrochemical, Nuclear, and Radiochemical Analysis of Nuclear-Grade Plutonium Metal

C759 Test Methods for Chemical, Mass Spectrometric, Spectrochemical, Nuclear, and Radiochemical Analysis of

Nuclear-Grade Plutonium Nitrate Solutions

C859 Terminology Relating to Nuclear Materials

C1009 Guide for Establishing and Maintaining a Quality Assurance Program for Analytical Laboratories Within the Nuclear Industry

C1168 Practice for Preparation and Dissolution of Plutonium Materials for Analysis

C1592/C1592M Guide for Making Quality Nondestructive Assay Measurements

E3376 Practice for Calibration and Usage of Germanium Detectors in Radiation Metrology for Reactor Dosimetry

2.2 U.S. Nuclear Regulatory Commission Regulatory Guides:³

Regulatory Guide 5.9, Rev. 2—Guidelines for Germanium Spectroscopy Systems for Measurement of Special Nuclear Material

Regulatory Guide 5.53, Rev. 1—Qualification, Calibration, and Error Estimation Methods for Nondestructive Assay³

3. Terminology

3.1 Except as otherwise defined herein, definitions of terms are as given in Terminology **C859**.

4. Summary of Test Method

4.1 An aliquot of the sample that contains about 10 ng to 100 ng of ^{241}Am is analyzed by measuring the intensity of the characteristic 59.5 keV gamma ray emitted by ^{241}Am .

4.2 Multiple sample geometries may be used if an appropriate calibration for each geometry is made.

4.3 The sample geometry must be reproducible. This includes the physical characteristics of the sample container, the positioning of the sample, and the volume of sample viewed by the gamma-ray detector.

4.4 Electronic corrections are made, if required, for the effects of pulse pile-up and dead time losses due to the activity of the sample. The necessity of dead time and pulse pile-up corrections can be reduced by sample dilution to control count rates.

¹ This test method is under the jurisdiction of ASTM Committee **C26** on Nuclear Fuel Cycle and is the direct responsibility of Subcommittee **C26.05** on Methods of Test.

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² For referenced ASTM standards, visit the ASTM website, www.astm.org, or contact ASTM Customer Service at service@astm.org. For *Annual Book of ASTM Standards* volume information, refer to the standard's Document Summary page on the ASTM website.

³ Available from U.S. Nuclear Regulatory Commission, One White Flint North, 11555 Rockville Pike, Rockville, MD 20852. Also through www.nrc.gov.

4.5 A correction is made for the contribution to the 59.5 keV intensity due to gamma rays produced in the decay of ^{237}U .

4.6 The relationship between the measured gamma-ray intensity and the ^{241}Am content is determined by the use of appropriate standards.

5. Significance and Use

5.1 This test method allows the determination of ^{241}Am in a plutonium solution without separation of the americium from the plutonium. It is generally applicable to any solution containing ^{241}Am .

5.2 The ^{241}Am in solid plutonium materials may be determined when these materials are dissolved (see Practice C1168).

5.3 When the plutonium solution contains unacceptable levels of fission products or other materials, this method may be used following a tri-n-octylphosphine oxide (TOPO) extraction, ion exchange or other similar separation techniques (see Test Methods C758 and C759).

5.4 This test method is less subject to interferences from plutonium than alpha counting since the energy of the gamma ray used for the analysis is better resolved from other gamma rays than the alpha particle energies used for alpha counting.

5.5 The minimal sample preparation reduces the amount of sample handling and exposure to the analyst.

5.6 This test method is applicable only to homogeneous solutions. This test method is not suitable for solutions containing solids.

5.7 Solutions containing ^{241}Am at concentrations as little as 1×10^{-5} g/L may be analyzed using this method. The lower limit depends on the detector used and the counting geometry. Solutions containing high concentrations may be analyzed following an appropriate dilution.

6. Interferences

6.1 The presence of other radioactive nuclides in the sample or in the vicinity of the detector may produce interferences. These may be due to the Compton scattering of high energy gamma rays which contribute to the background in the region of interest or from gamma rays with energies close to the energies used for the analysis.

6.2 The presence of ^{237}U will interfere if a correction is not applied. This interference will lead to an over estimation of the amount of ^{241}Am present. This interference is especially pronounced in plutonium from which the americium has recently been separated.

6.3 The presence of radioactive materials in the vicinity of the gamma-ray detector which are not in the sample may create interferences if detector shielding is not adequate. These interferences may be due to the Compton scattering of high energy gamma rays which contribute to the background in the region of interest or from gamma rays with energies close to the energies used for the analysis.

7. Apparatus

7.1 *High-Resolution Gamma Ray Counting System*—A high resolution gamma-ray counting system is required. General

guidelines for the selection of detectors and signal processing electronics are discussed in NRC Regulatory Guide 5.9. Data acquisition systems are addressed in NRC Regulatory Guide 5.9. This system should include the following items as a minimum.

7.1.1 *Germanium Photon Detector with Integral Preamplifier*—A coaxial type detector should typically have a full width at half maximum resolution of 850 eV or less at 122 keV and 2.0 keV or less at 1332 keV. A planar type detector should typically have a full width at half maximum resolution of 600 eV or less at 122 keV. Consideration should be given to the use of a high efficiency detector to enhance the ability to analyze low levels of americium.

7.1.2 *High Voltage Power Supply*—A high voltage power supply with voltage range and current output compatible with the detector selected is required. It is desirable that the voltage output be continuously adjustable.

7.1.3 *Nuclear Spectroscopy Amplifier*—Select a nuclear spectroscopy amplifier with pulse shaping, baseline restoration, and pulse pile-up rejection circuitry.

7.1.4 *Multichannel Pulse Height Analyzer (MCA)*—Select an MCA with a minimum of 2048 channels. It is desirable that the MCA be compatible with computerized operations so that data acquisition and analysis may be automated. The analog to digital converter (ADC) associated with the MCA should have a clock rate of at least 100 MHz and the capability of digitizing the input voltage range into a minimum of 2048 channels (other types of ADC's which provide equivalent capabilities can be used). The ADC should also have dead time and pulse pile-up correction capabilities.

7.2 *Sample Holder*, incorporating shielding to limit the interferences from background radiation sources, is required. Collimation to restrict the view of the detector to a portion of the sample may be required. The sample holder may incorporate more than one sample position. The sample holder shall provide reproducible positioning for each sample position so that a consistent volume or portion of the sample is viewed by the detector.

7.3 *Sample Vials* of sufficient volume to contain the desired sample as described in 10.2 are required. The sample vials should be made of low density materials and have reproducible dimensions such as wall thickness and internal diameter. Vials with identical dimensions should be used for samples and standards.

8. Hazards

8.1 This standard involves work with nuclear materials. The unique hazards and controls required to conduct the work contained in this standard from a safety, environmental, and security standpoint is the responsibility of the facility operators, in compliance with any applicable regulations. Any information given for handling these types of materials contained herein is meant only as a guide for consideration of the user and not a requirement.

8.2 Solutions and solids containing radioactive materials represent a potential for high radiation exposure to personnel handling them. Appropriate sample shielding, sample handling